Production of purified ⁸⁵Sr solution

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Since 2007, we have distributed purified radioisotopes such as 65 Zn, 88 Y, and 109 Cd to the general public.¹⁾ After the Fukushima Dai-ichi Nuclear Power Plant accident in 2011, the demand for 85 Sr solutions having a high specific radioactivity has been growing. In this work, we investigated the production of 85 Sr in the 85 Rb(d,2n) 85 Sr reaction using a 24-MeV deuteron beam from the RIKEN AVF cyclotron. We also studied a chemical procedure to obtain a purified 85 Sr solution.

⁸⁵Sr ($T_{1/2}$ = 64.853 d) was produced by irradiating an RbCl disk (Sigma-Aldrich; chemical purity: > 99.99%; thickness: 500 mg cm⁻²) of natural isotopic abundance with 24-MeV deuterons. The average beam intensity was 159 nA. The irradiation time was 17 min. 85 Sr was chemically separated in accordance with the scheme shown in Fig. 1. The irradiated RbCl target was dissolved in 2 mL of H₂O and 2 mL of 8 M HNO₃. After evaporating the solution almost to dryness, the residue was again dissolved in 2 mL of 8 M HNO₃. The resulting solution was evaporated to dryness to remove chloride ions, and the residue was dissolved in 4 mL of 8 M HNO₃ and loaded onto a reversed-phase extraction chromatography column (ϕ 5 mm×50 mm height) packed with Sr Resin (Eichrom; 100-150 mesh). The column was then washed with 12 mL of 8 M HNO₃. In this process, 85Sr was absorbed on the Sr Resin, and the target material of Rb was completely eluted, as traced with byproducts of ⁸⁴Rb and ⁸⁶Rb. ⁸⁵Sr was then eluted with 8 mL of 0.05 M HNO₃. The eluent was evaporated to dryness, and the residue was dissolved in 1 mL of concentrated HCl (c. HCl). After evaporating the solution almost to dryness, the residue was dissolved in 2 mL of 0.1 M HCl and loaded onto a column (ϕ 5 mm×40 mm height) packed with a cation-exchange resin (Dowex 50W×8; 200-400 mesh). The column was then washed with 3 M HCl. ⁸⁵Sr was eluted with 6 M HCl. The activity of ⁸⁵Sr was determined through γ -ray spectrometry using a calibrated Ge detector.

The γ -ray spectra of the produced ⁸⁵Sr are shown in Fig. 2. The produced activity of ⁸⁵Sr was 145 kBq, and the radionuclidic purity was > 99.9%. The production yield of ⁸⁵Sr under the present experimental condition was about 3 MBq μ A⁻¹ h⁻¹. The chemical yield was 89%.

The chemical impurity in the purified solution will be evaluated by using ICP MS for a control sample, which was treated using the same procedure as that used for the irradiated sample, in further studies.

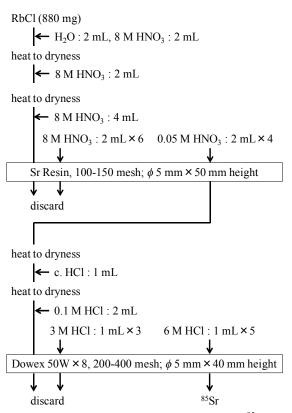


Fig. 1. The chemical separation procedure of ⁸⁵Sr from the irradiated RbCl target employed in the present study.

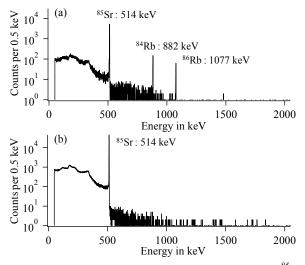


Fig. 2. The measured γ -ray spectra of the produced ⁸⁵Sr. (a) and (b) were obtained before and after the chemical separation, respectively.

References

1) T. Kambara et al.: a separate paper in this issue.

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